ELSEVIER

Nuclear Engineering and Design 159 (1995) 273–275

Nuclear Engineering and Design

Contents of Volume 159

CONTENTS OF ISSUE 159(1)

Extended and Updated Selected Papers from the 19th MPA Seminar, Stuttgart, Germany, 7–8 October, 1993

Preface ix

Operational monitoring in German nuclear power plants, *A. Seibold, J. Bartonicek and H. Kockelmann* 1

Determination of limits for smallest detectable and largest subcritical leakage cracks in piping systems, *R. Bieselt, B. Kuckartz and M. Wolf* 29

Estimating the tensile properties of an austenitic cladding tube steel following corrosion by uranium dioxide, simulated fission products and neutron exposure, *L. Schäfer* 43

The LBB project for the new generation of WWER plants and lessions learned from the WWER 440 model 230 LBB project, *J. Žd'árek, L. Pečínka and F. Kašpar* 55

Comparative analyses concerning integrity of a VVER-440 reactor pressure vessel, *J. Sievers, X. Liu, P. Rajamäki, H. Talja and H. Raiko* 63

Descriptive characteristics of different types of test for irradiation embrittlement, *K. Wallin, M. Valo, R. Rintamaa, K. Törrönen and R. Ahlstrand* 69

Irradiation devices and irradiation programmes at the high flux reactor (HFR) Petten for the investigation of the irradiation behaviour of stainless steels, *J.F.W. Markgraf, G.P. Tartaglia and G. Tsotridis* 81

Fatigue monitoring and life cycle analysis of a recuperative heat exchanger in the primary circuit, *B. Lucht, D. Humair and U. Blumer* 91

Instructions to Contributors 101

CONTENTS OF ISSUE 159(2–3)

Extended and Updated Selected Papers from Jahrestagung Kerntechnik '93, Cologne, Germany, 25–27 May, 1993

Preface xi

Decommissioning of nuclear installations in Russia and the other countries of the CIS, *S. Gavrilov and V. Kremnev* 105

Technical Engineering Note
Aspects on decommissioning of the Greifswald nuclear power plant, *B. Volkmann and U. Löschhorn* 117

Hard chemical decontamination of steam generator tube bundles, *M. Dubourg* 123
Influence of the depth position on the neutron embrittlement of the VVER reactor pressure vessel steel 15Cr2MoV(A). Consequences for the assessment of reactor safety, *J. Böhmert, F. Bergner, M. Grosse and H.-W. Viehrig* 131

Technical Engineering Note
Modelling operator rediagnosis during an SGTR event, *I. Parzer, S. Petelin and B. Mavko* 143

Information and signalling systems for FRG-1 and FRG-2, *K.-H. Blom, W. Krull and K.-R. Levknecht* 153

Technical Engineering Notes
Mobile system for the management of fuel assemblies from research reactors, *B. Löscher, H. Quaas and W. Hieronymus* 163
Analysis of operator actions during the accident caused by total loss of secondary heat sink, *S. Petelin, B. Mavko and O. Gortnar* 169
Inspections of pressure vessel head penetrations, *F. D'Annucci, G. Bäro and A. Assice* 177
Surveillance of nuclear-thermohydraulic stability in BWR operation, *R. Seepolt and O. Lorenz* 183

Safety requirements for the design of large containers for decommissioning waste, *K. Tunaboylu* 189

Technical Engineering Notes
Experimental investigations on a fuel element transportation vessel for analysis of mechanical loads, *R. Diersch, N. Ulrich, R. Hüggenberg, K. Janberg and A. Ettemeyer* 197
Efficiency of Monte Carlo perturbation calculations, *E. Seifert* 203
Drying medium level liquid waste concentrates, *S. Kunze and K. Gompper* 209
Investigations in the zero power reactor RAKE for an effective neutronic coupling of water-surrounded beam tubes, *G. Hüttel and Chr. Reiche* 213
Modelling of radioactive material release from waste packages in case of a fire event, *H.P. Berg and D. Gründler* 217
Calculation of the void reactivity of CANDU lattices using the SCALE code system, *J. Valkó, S. Fehér, J.E. Hoogenboom and J. Slobben* 225
PROMAX/PC — computer code for computing the maximum activity of radioactive waste packages and design of gamma-shielding, *W. Hauser, W. Wenzel and R. Müller* 233

Investigations concerning the semihydrostatic emplacement of waste packages in vertical boreholes, *W. Feuser* 239

Technical Engineering Note
ESAP, an easy-to-use expert system for the systematic analysis of operator actions within the scope of probabilistic risk assessment, *G. Degen, J. Mertens and B. Reer* 259

Analysis of rod-drop measurements at a VVER-440 reactor, *J. Krell, B. Mängel and H. Rebohm* 265

Contents of Volume 159 273
Author Index of Volume 159 277
Subject Index of Volume 159 281
Instructions to Contributors 283